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**A Programmatic Metamodel-Driven Modeling Paradigm for MCNP**

MCNP is a widely used and well validated Monte Carlo radiation transport code applicable for a plethora of reactor physics problems. Despite being a powerful simulation tool, MCNP provides no programmatic interface through which to automate execution, transform models, or support user-defined logic and abstractions. To address this need, we have developed an intuitive yet full-featured MCNP Application Program Interface (API) in Python, named `mcnpy`. Moreover, to guarantee our reading, writing, and modeling capabilities remain self-consistent (and to render the huge scope of the MCNP API manageable) we have adopted a strategy of model-driven software development described in previous work. As `mcnpy` is primarily a Python wrapper around a codebase automatically generated through this strategy, this ensures `mcnpy` too remains consistent with our MCNP (meta)model and maintainable. With `mcnpy`, users can freely read, write, and modify any syntactically valid MCNP input file regardless of its origin. These capabilities allow users to automate complicated tasks like nonlinear optimization and model translation. As examples, this work demonstrates the use of `mcnpy` to find the critical radius of a plutonium sphere and to translate a 9,000+ line MCNP input file into a corresponding OpenMC model.