





Nuclear Energy University Programs (NEUP) Fiscal Year (FY) 2019 RC-5 High Temperature Gas Reactors & RC-6: Fluoride Salt Cooled High Temperature Reactors

RC-5 High Temperature Gas Reactor

- FY19: During a primary coolant boundary break leading to depressurization of primary loop, cavity air entering the pressure vessel is a complex function of:
 - » Primary helium inventory
 - » Break location
 - » Break size
 - » Break orientation
 - » Venting pathways
- Previous experiment sponsored by the Department of Energy and NGNP Alliance yielded preliminary results and insights but a full parametric study has yet to be performed
- All experiments <u>MUST</u> be performed to NQA-1 standards.

Federal POC – Diana Li: Diana.LI@nuclear.energy.gov National Technical Director – Hans Gougar: Hans.Gougar@inl.gov

RC-5 High Temperature Gas Reactor

- FY19 interested in spatial distribution of air and helium in each reactor building cavity during the blowdown.
- Proposed projects should investigate:
 - Concentration of oxygen in the vessel due to various small and medium-sized breaks
 - Orientation of breaks,
 - Alternate ventilation pathways.
- Principal Investigators are encouraged to:
 - Consider recommendations of NGNP Alliance.
 - Consult with US-based HTGR vendors (Framatome, X-Energy, USNC) to refine the experiment design and test matrix.
- General Atomics 350 MWt MHTGR should be used as basis for scaling experimental facility.

RC-6: Fluoride Salt Cooled High Temperature Reactors

- The Fluoride Salt-cooled High Temperature Reactor (FHR) focus area seeks to address <u>one</u> of two areas identified by industry as gaps
- All results must be traceable via an NQA-1 process

Federal POC – Diana Li: Diana.LI@nuclear.energy.gov Technical POC – David Holcomb: holcombde@ornl.gov

RC-6: Fluoride Salt Cooled High Temperature Reactors

1) Used FHR pebble fuel storage and handling

- Seeking evaluation of technologies that support safe and effective used fuel pebble handling and passive cooling prior to dry storage
 - What type of system should be used to keep used FHR fuel pebbles cool until decay heat is low enough for dry storage?
 - How will the pebbles be transferred to this system?
 - What will the fissile material inventory be during the storage period?
 - What will be the necessary storage times to meet requirements for dry storage?
 - What will be the storage configuration of pebbles?
 - How can broken or cracked pebbles be safely stored?
 - Ensure sufficient capacity to store pebbles for the entire lifetime of the FHR.
 - Evaluations should also identify any necessary modifications to existing used fuel storage/transport casks for storage of FHR pebble bed fuel.

RC-6: Fluoride Salt Cooled High Temperature Reactors

2) Experimental validation of passive decay heat removal technology

- Demonstrate a Direct Reactor Auxiliary Cooling System (DRACS) system
 - Reduced capacity, DRACS-type natural circulation decay heat rejection system
 - Use prototypic materials and temperatures
 - Include the ability to simulate initial loading and startup, normal reactor operations, transition to accident conditions, and extended duration operation
 - Confirm the system will sufficiently remove heat without salt freezing as cooling requirements reduce over time
 - Include tritium release mitigation mechanisms