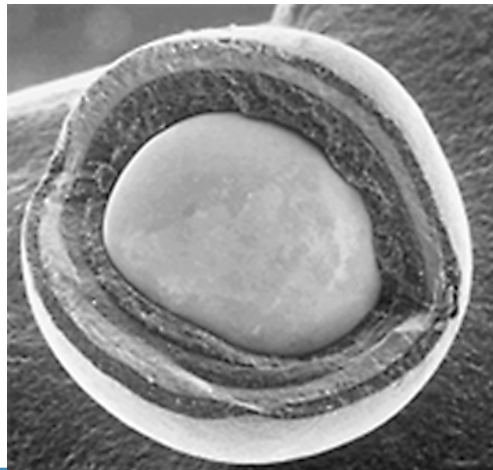


Nuclear Energy University Program FY18 CINR FOA
Program Supporting: Nuclear Reactor Technologies
Materials for Advanced Reactor Technologies (RC-1)

Oxidation Behavior of HTGR TRISO Fuel Materials (RC-1.3)

Dr. Madeline Anne Feltus
DOE Advanced Gas Reactor
TRISO Fuels Program Manager

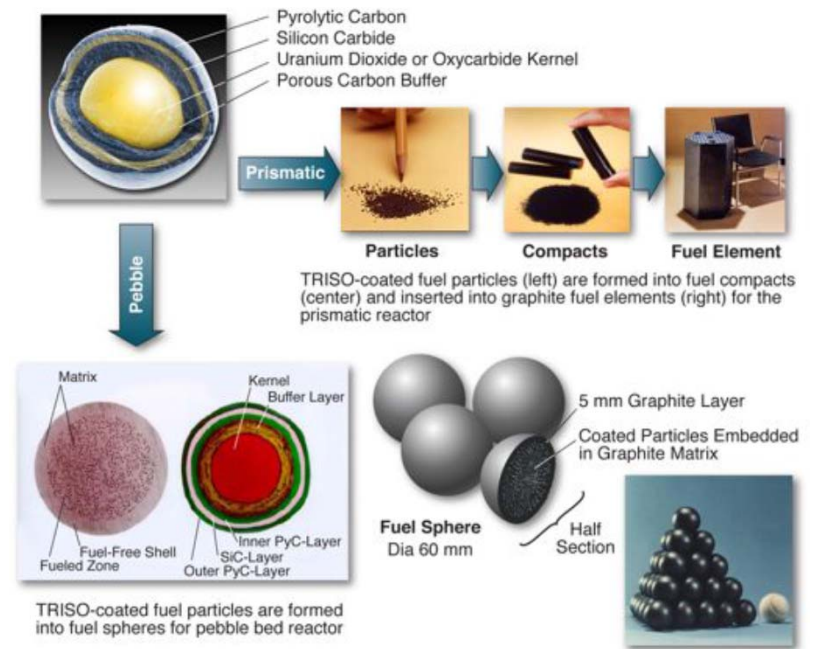


Motivation for getting oxidation behavior in TRISO fuels:

- HTGRs use pure helium as reactor coolant, but trace quantities of oxidants (air, moisture, CO, CO₂) may be present during normal operations
- If a steam generator tube leak occurs, large amounts of moisture can be introduced into the helium coolant and reactor core.
- Air can be introduced during some accident conditions, e.g., depressurization of the helium cooling loop from coolant line break.
- Understanding the effects of oxidants (air, moisture, CO, CO₂) on TRISO fuel integrity and fission product transport is essential for HTGR safety analysis.
- Fuel particle, compact matrix oxidation **experimental data** are needed to model HTGR core accident behavior.

Eligible to Lead:
Maximum funding
Duration:

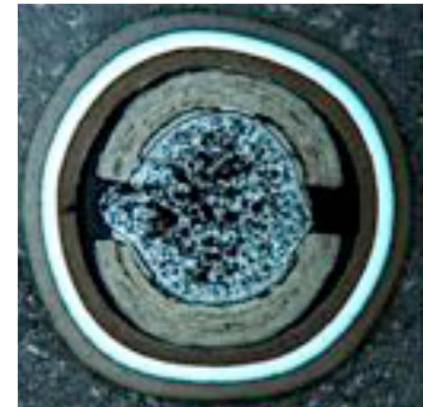
Universities Only
\$800,000
Up to 3 years



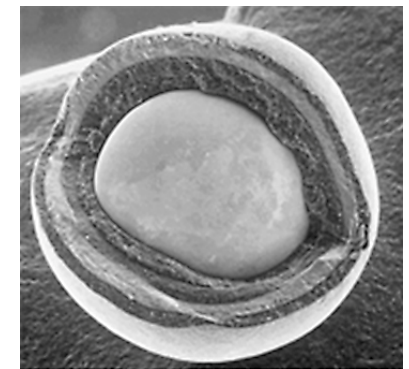
Proposals MUST:

- **Experimentally** test moisture oxidation of graphitic matrix and/or air or moisture oxidation of SiC in the passive to active transition regime
- Use the most prototypic available, e.g., un-irradiated and irradiated TRISO, SiC, matrix materials available from AGR TRISO program sample archives
- Emphasize carefully designed experiments that can be used to compare with current, future, or develop new oxidation models
- Produce useful results that can simulate oxidation of the fuel during air or moisture ingress accidents and predict partial pressures of oxidants transported to the TRISO fuel particles.

Proposals **MUST NOT** focus only on developing computational models or correlations, without performing any new oxidation experiments.

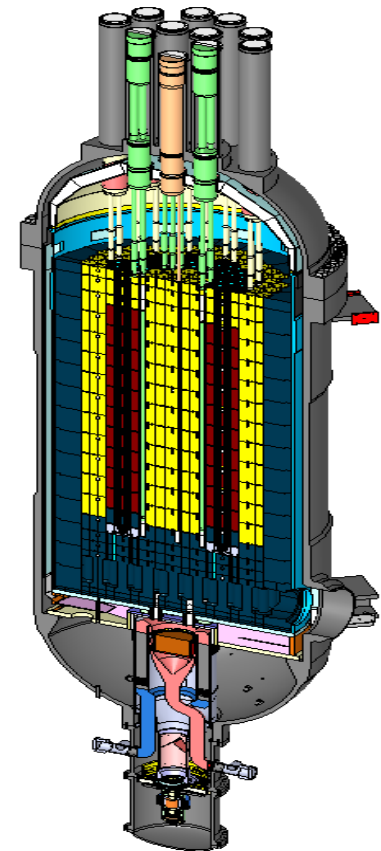


Irradiated
TRISO



Proposals should:

- Consider the effect of irradiation on SiC and matrix oxidation properties using irradiated specimens
- Focus on oxidation **experiments** using the results to develop correlations for air and/or moisture ingress accidents.
- Use realistic ranges for HTGR temperatures, gas flow rates, core geometries, oxidant partial pressures, etc.
- Provide data about the level of SiC damage that could potentially occur during air/moisture ingress accidents.
- Provide useful information about the transition from active to passive (and passive to active) oxidation of chemical vapor deposition (CVD) SiC in TRISO particles at the temperatures of interest ($\sim 1000^{\circ}\text{C}$ - 1600°C) under relevant atmospheres, e.g., those containing O_2 or, especially, H_2O vapor



Proposals must NOT repeat earlier tests performed by NEUP grants, ART, AGR TRISO, and AGC graphite programs:**

- **Chronic oxidation of nuclear graphite (only) during air/moisture ingress conditions.**
- **Previous TRISO fuel matrix material (graphitic material composed of many graphite types, carbonized phenolic resin) experiments in air**
- **Recent tests on the silicon carbide layer of surrogate (non-uranium bearing) coated particles at high temperatures (~1600°C) in high steam or air partial pressures**
- **SiC oxidation in air at high oxidant partial pressures, high temperatures**

****See INL Advanced Reactors Technology, AGR TRISO fuels, NGNP, NEUP websites at:**

<https://art.inl.gov/default.aspx>; <https://neup.inl.gov>

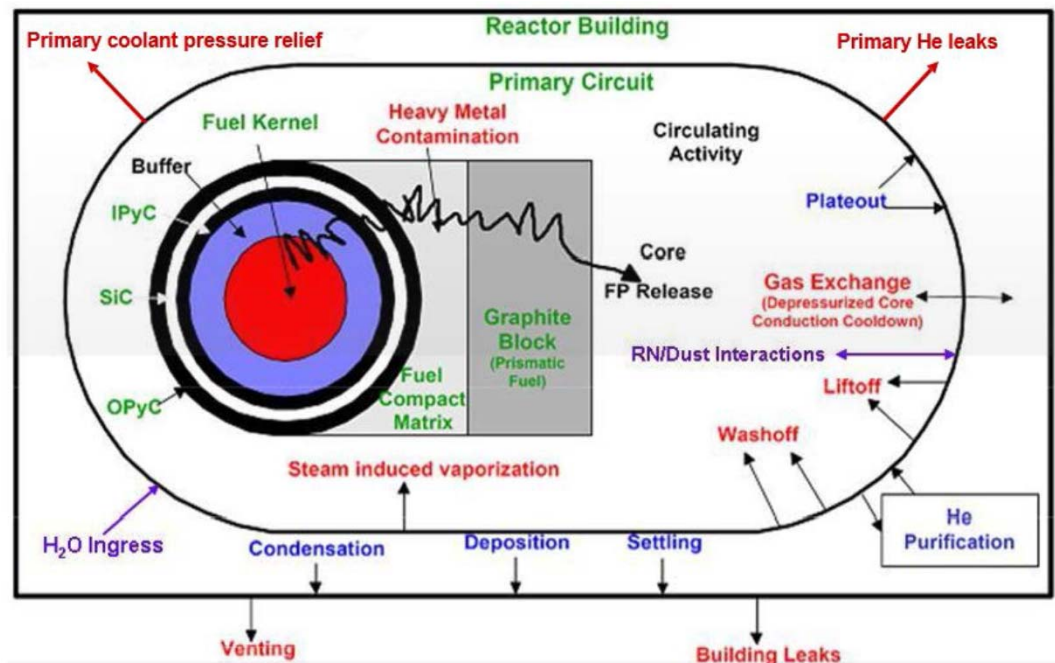
ORNL CCTF



INL FACS Furnace

Proposals could perform new separate effects experiments with air/moisture ingress oxidation conditions not tested so far:

- Oxidation of fuel particles and matrix material in water vapor with varying oxidant partial pressures to determine reaction kinetic parameters behavior
- Determine the effect of TRISO fuel matrix burn-off on the rate of oxidation and reaction kinetic effects
- SiC oxidation at lower oxidant (moisture, air) partial pressures where active/passive oxidation transition occurs.
- TRISO graphitic matrix oxidation under varying moisture/water vapor conditions



Proposals may:

- Use in-situ university laboratories for surrogate, non-radioactive specimen and material experiments.
- Demonstrate experiments, test rig configurations that may be practical for using radioactive isotopes and materials in hot cells.
- Use existing specimens from Advanced Gas Reactor (AGR) TRISO fuel experiments and/or Advanced Graphite Creep tests at NSUF locations for hot-cell PIE, SEM, TEM, FIB microscopy, etc.

Suggestions:

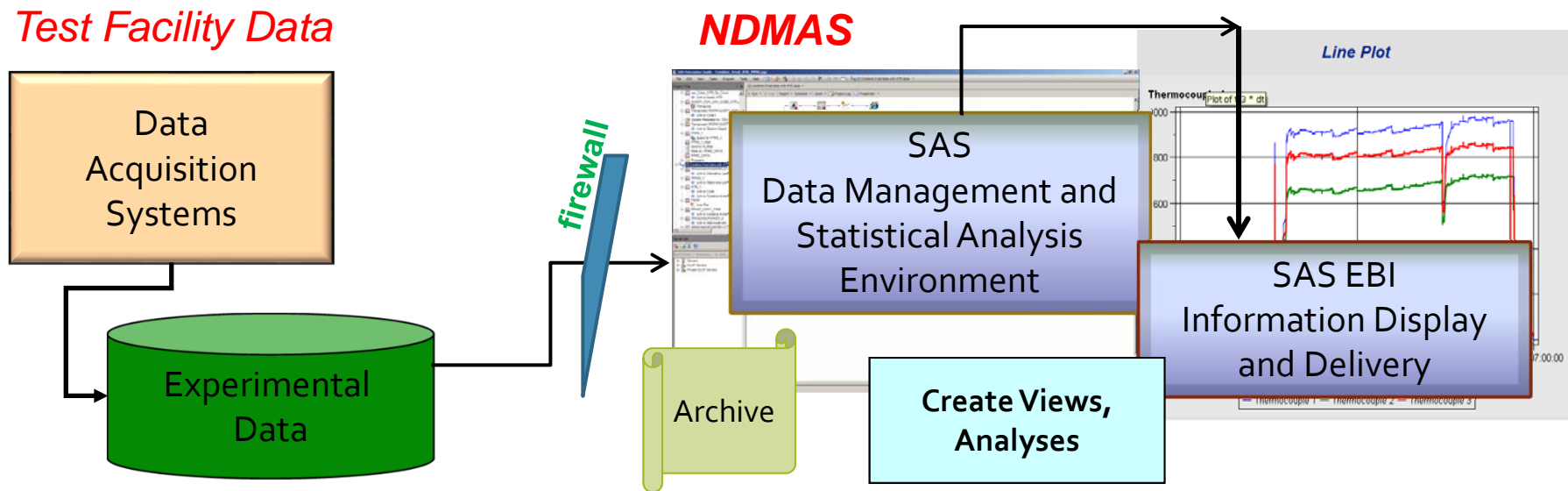
- Coordinate with AGR TRISO Fuel Program staff to get appropriate irradiated, un-irradiated specimens.
- Interaction with HTGR fuel and reactor designers on their system requirements is highly encouraged.



Quality Assurance Compliance

Quality Assurance and Data Retention:

- Data collection, experiments, data validation, and verification may require compliance with NQA-1 2009 and 2009 NRC accepted paragraphs.
- Archiving data and simulation results in the INL Nuclear Data Management and Analysis System (NDMAS) may be required



Interested university applicants may contact:

• ***Federal POC:***

Dr. Madeline Feltus

DOE, AGR TRISO Program Manager

madeline.feltus@nuclear.energy.gov

• ***Technical POC:***

Dr. Paul Demkowicz

INL, AGR TRISO Technical Manager

paul.demkowicz@inl.gov

